National Institute of Standards & Technology
Certificate

Standard Reference Material 4425L, Lot 8
Samarium-153 Radioactivity Standard

This Standard Reference Material (SRM) consists of radioactive samarium-153 chloride, non-radioactive samarium chloride, and hydrochloric acid dissolved in 5 mL of distilled water. The solution is contained in a flame-sealed NIST borosilicate-glass ampoule. The SRM is intended for the calibration of ionization chambers and solid-state gamma-ray spectrometry systems.

Radiological Hazard

The SRM ampoule contains samarium-153 with a total activity of approximately 30 MBq. Samarium-153 decays by beta-particle emission. During the decay process, X-rays and gamma rays with energies from approximately 6 keV to 800 keV are emitted. Most of these photons escape from the SRM ampoule and can represent a radiation hazard. Approximate unshielded dose rates at several distances (as of the reference time) are given in note [a]*. Appropriate shielding and/or distance should be used to minimize personnel exposure. The SRM should be used only by persons qualified to handle radioactive material.

Chemical Hazard

The SRM ampoule contains hydrochloric acid with a concentration of approximately 1 mole per liter of water. The solution is corrosive and represents a health hazard if it comes in contact with eyes or skin. If the ampoule is to be opened to transfer the solution, the recommended procedure is given on page 2. The ampoule should be opened only by persons qualified to handle both radioactive material and strong acid solution.

Storage and Handling

The SRM should be stored and used at a temperature between 5 °C and 65 °C. The solution in an unopened ampoule should remain stable and homogeneous until at least July 2003. The ampoule (or any subsequent container) should always be clearly marked as containing radioactive material. If the ampoule is transported it should be packed, marked, labeled, and shipped in accordance with the applicable national, international, and carrier regulations. The solution in the ampoule is a dangerous good (hazardous material) because of both the radioactivity and the strong acid.

Preparation

This Standard Reference Material was prepared in the Physics Laboratory, Ionizing Radiation Division, Radioactivity Group, B.M Coursey, Acting Group Leader. The overall technical direction and physical measurements leading to certification were provided by D.B. Golas and O.T. Palabrica, Nuclear Energy Institute Research Associates. The support aspects involved in the preparation, certification, and issuance of this SRM were coordinated through the Standard Reference Materials Program by J.W.L. Thomas.

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*Notes and references are on pages 5 to 7.

Recommended Procedure for Opening the SRM Ampoule

1) If the SRM solution is to be diluted, it is recommended that the diluting solution have a composition comparable to that of the SRM solution.

2) Wear eye protection, gloves, and protective clothing and work over a tray with absorbent paper in it. Work in a fume hood. In addition to the radioactive material, the solution contains strong acid and is corrosive.

3) Shake the ampoule to wet all of the inside surface of the ampoule. Return the ampoule to the upright position.

4) Check that all of the liquid has drained out of the neck of the ampoule. If necessary, gently tap the neck to speed the process.

5) Holding the ampoule upright, score the narrowest part of the neck with a scribe or diamond pencil.

6) Lightly wet the scored line. This reduces the crack propagation velocity and makes for a cleaner break.

7) Hold the ampoule upright with a paper towel, a wiper, or a support jig. Position the scored line away from you. Using a paper towel or wiper to avoid contamination, snap off the top of the ampoule by pressing the narrowest part of the neck away from you while pulling the tip of the ampoule towards you.

8) Transfer the solution from the ampoule using a pycnometer or a pipet with dispenser handle. NEVER PIPETTE BY MOUTH.

9) Seal any unused SRM solution in a flame-sealed glass ampoule, if possible, to minimize the evaporation loss.

See also reference [4]*.

*Notes and references are on pages 5 to 7.

Certified values

Radionuclide	Samarium-153
Reference time	1400 EST, 23 July 2002
Massic activity of the solution [b]	5.890 MBq·g^{-1}
Relative expanded uncertainty (k=2)	1.5% [c] [d]
Solution mass	(mass ± 0.0003) g [e]
Solution density	(1.015 ± 0.002) g·mL^{-1} at 20 °C [e]*

Uncertified values

Physical Properties:	
Source description	Liquid in flame-sealed NIST borosilicate-glass ampoule
Ampoule specifications	Body outside diameter (16.5 ± 0.5) mm Wall thickness (0.60 ± 0.04) mm Barium content Less than 2.5% Lead-oxide content Less than 0.02% Other heavy elements Trace quantities

Chemical Properties:			
Solution composition	Chemical Formula	Concentration (mol·L^{-1})	Mass Fraction (g·g^{-1})
	H_2O	54	0.96
	HCl	1.1	0.04
	$SmCl_3$	1×10^{-4}	3×10^{-5}
	$^{153}SmCl_3$	2×10^{-9}	6×10^{-10}

Radiological Properties:	
Photon-emitting impurities	Europium-152: (0.4 ± 0.1) kBq·g^{-1} [e] [f] Europium-154: (2.1 ± 0.4) kBq·g^{-1} [e] [f] Europium-155: (0.4 ± 0.1) kBq·g^{-1} [e] [f] Europium-156: (5.7 ± 1.1) kBq·g^{-1} [e] [f]
Half lives used	Samarium-153: (46.284 ± 0.004) h [g] [5] Europium-152: (13.537 ± 0.006) a [g] [5] Europium-154: (8.593 ± 0.004) a [g] [5] Europium-155: (4.7611 ± 0.0013) a [g] [5] Europium-156: (15.19 ± 0.08) d [g] [5] Radium-226: (1600 ± 7) a [g] [5]
Calibration method and measuring instrument(s)	Pressurized "4π"γ ionization chamber A calibrated using a samarium-153 solution whose activity was determined by 4πβ liquid-scintillation counting. The samarium-153 liquid-scintillation detection efficiency was calculated using the CIEMAT/NIST method with hydrogen-3 as the detection-efficiency monitor. [h]

*Notes and references are on pages 5 to 7.

EVALUATION OF THE UNCERTAINTY OF THE MASSIC ACTIVITY [c]*

Input Quantity x_i, the source of uncertainty (and individual uncertainty components where appropriate)	Method Used To Evaluate $u(x_i)$, the standard uncertainty of x_i (A) denotes evaluation by statistical methods (B) denotes evaluation by other methods	Relative Uncertainty Of Input Quantity, $u(x_i)/x_i$, (%) [i]	Relative Sensitivity Factor, $\lvert \partial y/\partial x_i \rvert \cdot (x_i/y)$ [j]	Relative Uncertainty Of Output Quantity, $u_i(y)/y$, (%) [k]
PIC A net response per gram of SRM 4425L, measured relative to RRS50 [m]	Standard deviation of the mean for >100 repeated measurements (A)	0.03	1.0	0.03
PIC A net response for RRS50, measured relative to RRS3	Standard deviation of the mean for >100 repeated measurements (A)	0.09	1.0	0.09
PIC A net response per Bq of samarium-153 in solution, measured relative to RRS3	Standard deviation of the mean for >100 repeated measurements (A)	0.12	1.0	0.12
Activity used to calibrate PIC A net response per Bq of samarium-153 in solution	Standard uncertainty of the activity determined by $4\pi\beta$ liquid-scintillation counting (B)	0.73	1.0	0.73
Corrections for different solution density and carrier concentration	Estimated (B)	0.05	1.0	0.05
Half life of samarium-153 Half life of radium-226	Standard uncertainty of the half life (A)	0.009 [n] 0.44 [n]	0.02 [p] 0.006 [p]	0.0002 0.003
Gravimetric measurements	Estimated (B)	0.05	1.0	0.05
Live time [q]	Estimated (B)	0.05	1.0	0.05
PIC A charge collection	Estimated (B)	0.05	1.0	0.05
Source positioning	Estimated (B)	0.05	1.0	0.05
Photon-emitting impurities	Estimated (B) [r] Estimated (B) [r] Estimated (B) [r] Estimated (B) [r] Limit of detection (B) [s]	10 10 10 10 100.	0.001 0.004 0.0001 0.01 0.0005	0.01 0.04 0.001 0.10 0.05

Relative Combined Standard Uncertainty of the Output Quantity, $u_c(y)/y$, (%)   0.77

Coverage Factor, k x 2

Relative Expanded Uncertainty of the Output Quantity, U/y, (%)   1.5

*Notes and references are on pages 5 to 7.

NOTES

[a] The Sievert is the SI unit for dose equivalent. See reference [1]. One μSv is equal to 0.1 mrem.

Distance from Ampoule (cm):	1	30	100
Approximate Dose Rate (μSv/h):	300	3	<1

[b] **Massic activity** is the preferred name for the quantity activity divided by the total mass of the sample. See reference [1].

[c] The reported value, y, of massic activity (activity per unit mass) at the reference time was not measured directly but was derived from measurements and calculations of other quantities. This can be expressed as $y = f(x_1, x_2, x_3, \ldots x_n)$, where f is a mathematical function derived from the assumed model of the measurement process. The value, x_i, used for each input quantity i has a **standard uncertainty**, $u(x_i)$, that generates a corresponding uncertainty in y, $u_i(y) \equiv |\partial y/\partial x_i| \cdot u(x_i)$, called a **component of combined standard uncertainty** of y. The **combined standard uncertainty** of y, $u_c(y)$, is the positive square root of the sum of the squares of the components of combined standard uncertainty. The combined standard uncertainty is multiplied by a **coverage factor** of $k = 2$ to obtain U, the **expanded uncertainty** of y.

Since it can be assumed that the possible estimated values of the massic activity are approximately normally distributed with approximate standard deviation $u_c(y)$, the unknown value of the massic activity is believed to lie in the interval $y \pm U$ with a level of confidence of approximately 95 percent.

For further information on the expression of uncertainties, see references [2] and [3].

[d] The value of each standard uncertainty component, and hence the value of the expanded uncertainty itself, is a best estimate based upon all available information, but is only approximately known. That is to say, the "uncertainty of the uncertainty" is large and not well known. This is true for uncertainties evaluated by statistical methods (e.g., the relative standard deviation of the standard deviation of the mean for the massic response is approximately 50%) and for uncertainties evaluated by other methods (which could easily be over estimated or under estimated by substantial amounts). The unknown value of the expanded uncertainty is believed to lie in the interval $U/2$ to $2U$ (i.e., within a factor of 2 of the estimated value).

[e] The stated uncertainty is two times the standard uncertainty.

[f] Estimated limits of detection for photon-emitting impurities, as of 26 August 2002 (34 days after the reference time), expressed as massic photon emission rates, are:
8×10^1 s$^{-1} \cdot$g^{-1} for energies between 40 keV and 60 keV, and
2×10^1 s$^{-1} \cdot$g^{-1} for energies between 60 keV and 3000 keV, provided that the photons
are separated in energy by 4 keV or more from photons emitted in the decay of samarium-153.

[g] The stated uncertainty is the standard uncertainty.

[h] The relationship between the detection efficiency for samarium-153 and the detection efficiency for hydrogen-3 was computed using the CIEMAT/NIST method as embodied in the computer program EFFY4. See references [6, 7, 8]. The program computes the detection efficiency for each radionuclide based upon an assumed model.

[i]   Relative standard uncertainty of the input quantity x_i.

[j]   The relative change in the output quantity y divided by the relative change in the input quantity x_i. If $|\partial y/\partial x_i| \cdot (x_i/y) = 1.0$, then a 1% change in x_i results in a 1% change in y. If $|\partial y/\partial x_i| \cdot (x_i/y) = 0.05$, then a 1% change in x_i results in a 0.05% change in y.

[k]   Relative component of combined standard uncertainty of output quantity y, rounded to two decimal places. The relative component of combined standard uncertainty of y is given by $u_i(y)/y \equiv |\partial y/\partial x_i| \cdot u(x_i)/y = |\partial y/\partial x_i| \cdot (x_i/y) \cdot u(x_i)/x_i$. The numerical values of $u(x_i)/x_i$, $|\partial y/\partial x_i| \cdot (x_i/y)$, and $u_i(y)/y$, all dimensionless quantities, are listed in columns 3, 4, and 5, respectively. Thus, the value in column 5 is equal to the value in column 4 multiplied by the value in column 3. The input quantities are independent, or very nearly so. Hence the covariances are zero or negligible.

[m]   The response of pressurized ionization chamber A (PIC A) is determined from measurement of the time required to collect a given amount of charge on a stable fixed capacitor. All of the response measurements in the NIST pressurized ionization chambers are made relative to the response of one or more artifact standards. These artifact standards consist of microgram quantities of aged radium-226 in small welded stainless-steel capsules. These capsules are encapsulated in plastic rods whose dimensions are similar to those of the standard NIST ampoule. The artifact standards are called **Radium Reference Sources** and are designated as RRSx, where x is the nominal mass (in micrograms) of radium-226 in the capsule.

[n]   The relative standard uncertainty of $\lambda \cdot t$ is determined by the relative standard uncertainty of λ (i.e., of the half life). The relative standard uncertainty of t is negligible.

[p] $|\partial y/\partial x_i| \cdot (x_i/y) = |\lambda \cdot t|$

[q]   The live time is determined by counting the pulses from a gated crystal-controlled oscillator.

[r]   The standard uncertainties given are for the detected impurities. $|\partial y/\partial x_i| \cdot (x_i/y) = \{(\text{response per Bq of impurity})/(\text{response per Bq of samarium-153})\} \cdot \{(\text{Bq of impurity})/(\text{Bq of samarium-153})\}$.

[s]   The standard uncertainty for each undetected impurity that might reasonably be expected to be present is estimated to be equal to the estimated limit of detection for that impurity, i.e. $u(x_i)/x_i = 100\%$. $|\partial y/\partial x_i| \cdot (x_i/y) = \{(\text{response per Bq of impurity})/(\text{response per Bq of samarium-153})\} \cdot \{(\text{Bq of impurity})/(\text{Bq of samarium-153})\}$. Thus $u_i(y)/y$ is the relative change in y if the impurity were present with a massic activity equal to the estimated limit of detection.

REFERENCES

[1]   International Organization for Standardization (ISO), *ISO Standards Handbook - Quantities and Units*, 1993. Available from Global Engineering Documents, 12 Inverness Way East, Englewood, CO 80112, U.S.A. Telephone 1-800-854-7179.

[2]   International Organization for Standardization (ISO), *Guide to the Expression of Uncertainty in Measurement*, 1993 (corrected and reprinted, 1995). Available from Global Engineering Documents, 12 Inverness Way East, Englewood, CO 80112, U.S.A. Telephone 1-800-854-7179.

[3]   B. N. Taylor and C. E. Kuyatt, *Guidelines for Evaluating and Expressing the Uncertainty of NIST Measurement Results*, NIST Technical Note 1297, 1994. Available from the Superintendent of Documents, U.S. Government Printing Office, Washington, DC 20407, U.S.A.

[4]   National Council on Radiation Protection and Measurements Report No. 58, *A Handbook of Radioactivity Measurements Procedures*, Second Edition, 1985. Available from the National Council on Radiation Protection and Measurements, 7910 Woodmont Avenue, Bethesda, MD 20814 U.S.A.

[5]   Evaluated Nuclear Structure Data File (ENSDF), July 2002.

[6]   E. Garcia-Toraño and A. Grau Malonda, EFFY, A New Program to Compute the Counting Efficiency of Beta Particles in Liquid Scintillators, *Computer Phys. Comm.* **36** (1985) 307.

[7]   A. Grau Malonda, E. Garcia-Toraño, and J.M. Los Arcos, Liquid-Scintillation Counting Efficiency as a Function of the Figure of Merit for Pure Beta-Particle Emitters, *Int. J. Appl. Radiat. Isot.* **36** (1985) 157.

[8]   B.M. Coursey, W.B. Mann, A. Grau Malonda, E. Garcia-Toraño, J.M. Los Arcos, J.A.B. Gibson, and D. Reher, Standardization of Carbon-14 by 4πβ Liquid Scintillation Efficiency Tracing with Hydrogen-3, *Appl. Radiat. Isot.* **37** (1986) 403.