National Institute of Standards & Technology
Certificate

Standard Reference Material 4324B
Uranium-232 Radioactivity Standard

This Standard Reference Material (SRM) consists of radioactive uranium-232 nitrate (in equilibrium with its progeny) and nitric acid dissolved in 5 mL of distilled water. The solution is contained in a flame-sealed NIST borosilicate-glass ampoule. The SRM is intended for the calibration of alpha-particle counting instruments and for the monitoring of radiochemical procedures.

Radiological Hazard

The SRM ampoule contains uranium-232 with a total activity of approximately 200 Bq. Uranium-232 decays by alpha-particle emission. The progeny of uranium-232 have a total activity of approximately 1400 Bq and decay by alpha- and beta-particle emission. None of the alpha or beta particles escape from the SRM ampoule. During the decay process X-rays and gamma rays with energies from 10 keV to 4.5 MeV are also emitted. Most of these photons escape from the SRM ampoule but their intensities are so small that they do not represent a radiation hazard. Approximate unshielded dose rates at several distances (as of the reference time) are given in note [a]*. The SRM should be used only by persons qualified to handle radioactive material.

Chemical Hazard

The SRM ampoule contains nitric acid with a concentration of approximately 2 moles per liter of water. The solution is corrosive and represents a health hazard if it comes in contact with eyes or skin. If the ampoule is to be opened to transfer the solution, the recommended procedure is given on page 2. The ampoule should be opened only by persons qualified to handle both radioactive material and strong acid solution.

Storage and Handling

The SRM should be stored and used at a temperature between 5 °C and 65 °C. The solution in an unopened ampoule should remain stable and homogeneous until at least July 2012. The ampoule (or any subsequent container) should always be clearly marked as containing radioactive material. If the ampoule is transported it should be packed, marked, labeled, and shipped in accordance with the applicable national, international, and carrier regulations. The solution in the ampoule is a dangerous good (hazardous material) both because of the radioactivity and because of the strong acid.

Preparation

This Standard Reference Material was prepared in the Physics Laboratory, Ionizing Radiation Division, Radioactivity Group, B.M. Coursey, Acting Group Leader. The overall technical direction and physical measurements leading to certification were provided by L.L. Lucas of the Radioactivity Group. The support aspects involved in the preparation, certification, and issuance of this SRM were coordinated through the Standard Reference Materials Program by J.W.L. Thomas.

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*Notes and references are on pages 5 to 7.

Recommended Procedure for Opening the SRM Ampoule

1) If the SRM solution is to be diluted, it is recommended that the diluting solution have a composition comparable to that of the SRM solution.

2) Wear eye protection, gloves, and protective clothing and work over a tray with absorbent paper in it. Work in a fume hood. In addition to the radioactive material, the solution contains strong acid and is corrosive.

3) Shake the ampoule to wet all of the inside surface of the ampoule. Return the ampoule to the upright position.

4) Check that all of the liquid has drained out of the neck of the ampoule. If necessary, gently tap the neck to speed the process.

5) Holding the ampoule upright, score the narrowest part of the neck with a scribe or diamond pencil.

6) Lightly wet the scored line. This reduces the crack propagation velocity and makes for a cleaner break.

7) Hold the ampoule upright with a paper towel, a wiper, or a support jig. Position the scored line away from you. Using a paper towel or wiper to avoid contamination, snap off the top of the ampoule by pressing the narrowest part of the neck away from you while pulling the tip of the ampoule towards you.

8) Transfer the solution from the ampoule using a pycnometer or a pipet with dispenser handle. NEVER PIPETTE BY MOUTH.

9) Seal any unused SRM solution in a flame-sealed glass ampoule, if possible, to minimize the evaporation loss.

See also reference [4]*.

*Notes and references are on pages 5 to 7.

PROPERTIES OF SRM 4324B

Certified values

Radionuclide	Uranium-232 (in equilibrium with its progeny)
Reference time	1200 EST, 01 July 2002
Massic activity of the solution [b]*	38.22 Bq(U-232)$\cdot$g^{-1}
Relative expanded uncertainty (k=2)	0.80% [c] [d]
Solution mass	(5.321 $\pm$ 0.001) g [e]
Solution density	(1.064 $\pm$ 0.002) g$\cdot$mL^{-1} at 20 °C [e]*

Uncertified values

Physical Properties:			
Source description	Liquid in flame-sealed NIST borosilicate-glass ampoule		
Ampoule specifications	Body outside diameter (16.5 $\pm$ 0.5) mm Wall thickness (0.60 $\pm$ 0.04) mm Barium content Less than 2.5% Lead-oxide content Less than 0.02% Other heavy elements Trace quantities		

Chemical Properties:			
Solution composition	Chemical Formula	Concentration (mol$\cdot$L^{-1})	Mass Fraction (g$\cdot$g^{-1})
	H_2O	52	0.88
	HNO_3	2	0.12
	$^{232}UO_2(NO_3)_2$	2×10^{-10}	8×10^{-11}

Radiological Properties:	
Alpha-particle-emitting impurities	None detected [f]
Photon-emitting impurities	None detected [g]
Half lives used	Uranium-232: (68.9 $\pm$ 0.4) a [h] [5] Thorium-228: (1.9116 $\pm$ 0.0016) a [h] [5] Radium-224: (3.66 $\pm$ 0.04) d [h] [5] Radon-220: (55.6 $\pm$ 0.1) s [h] [5] Polonium-216: (0.145 $\pm$ 0.002) s [h] [5] Lead-212: (10.64 $\pm$ 0.01) h [h] [5] Bismuth-212: (60.55 $\pm$ 0.06) m [h] [5] Polonium-212: (299 $\pm$ 2) ns [h] [5] Thallium-208: (3.053 $\pm$ 0.004) m [h] [5]
Calibration method and measuring instrument(s)	NIST $4\pi(\alpha+\beta)$(LS)-γ-anticoincidence counting system.

*Notes and references are on pages 5 to 7.

EVALUATION OF THE UNCERTAINTY OF THE MASSIC ACTIVITY [c,d]*

Input Quantity x_i, the source of uncertainty (and individual uncertainty components where appropriate)	Method Used To Evaluate $u(x_i)$, the standard uncertainty of x_i (A) denotes evaluation by statistical methods (B) denotes evaluation by other methods	Relative Uncertainty Of Input Quantity, $u(x_i)/x_i$, (%) [i]	Relative Sensitivity Factor, $\lvert \partial y/\partial x_i \rvert \cdot (x_i/y)$ [j]	Relative Uncertainty Of Output Quantity, $u_i(y)/y$, (%) [k]
Extrapolated massic count rate, corrected for background, decay, and ingrowth of daughters	Standard deviation of the mean for 22 sets of repeated measurements on three samples (A)	0.02	1.0	0.02
Gravimetric measurements	Estimated (B)	0.10	1.0	0.10
Half life of uranium-232	Standard uncertainty of the half life (A)	0.58 [m]	0.0006 [n]	0.0004
Half life of thorium-228	Standard uncertainty of the half life (A)	0.08 [m]	0.12	0.01
Decay-scheme data	Estimated (B)	0.20	0.15	0.03
Live time [p]	Estimated (B)	0.10	1.0	0.10
Extrapolation of the LS(α+ß)-count-rate versus anticoincidence-gamma-ray-count-rate to zero anticoincidence-gamma-ray-count-rate	Estimated (B)	0.36	1.0	0.36
Alpha-particle-emitting impurities	Limit of detection (B) [q]	100.	0.001	0.10
Photon-emitting impurities	Limit of detection (B) [q]	100.	0.0001	0.01

Relative Combined Standard Uncertainty of the Output Quantity, $u_c(y)/y$, (%)	0.40
Coverage Factor, k	x 2
Relative Expanded Uncertainty of the Output Quantity, U/y, (%)	0.80

*Notes and references are on pages 5 to 7.

RELATIVE ACTIVITIES OF URANIUM-232 AND ITS PROGENY AT EQUILIBRIUM

Radionuclide	Half Life [h] [5]	Relative Activity [r,s]
Uranium-232	(68.9 ± 0.4) years	1.000 000
Thorium-228	(1.9116 ± 0.0016) years	1.028 536
Radium-224	(3.66 ± 0.04) days	1.028 686
Radon-220	(55.6 ± 0.1) seconds	1.028 686
Polonium-216	(0.145 ± 0.002) seconds	1.028 686
Lead-212	(10.64 ± 0.01) hours	1.028 704
Bismuth-212	(60.55 ± 0.06) minutes	1.028 706
Polonium-212	(299 ± 2) nanoseconds	0.658 989
Thallium-208	(3.053 ± 0.004) minutes	0.369 717
Lead-208	Stable	-

NOTES

[a] The Sievert is the SI unit for dose equivalent. See reference [1]. One μSv is equal to 0.1 mrem.

Distance from Ampoule (cm): 1   30   100

Approximate Dose Rate (μSv/h): <0.1   -   -

[b] **Massic activity** is the preferred name for the quantity activity divided by the total mass of the sample. See reference [1].

[c] The reported value, y, of massic activity (activity per unit mass) at the reference time was not measured directly but was derived from measurements and calculations of other quantities. This can be expressed as $y = f(x_1, x_2, x_3, \ldots x_n)$, where f is a mathematical function derived from the assumed model of the measurement process. The value, x_i, used for each input quantity i has a **standard uncertainty**, $u(x_i)$, that generates a corresponding uncertainty in y, $u_i(y) \equiv |\partial y / \partial x_i| \cdot u(x_i)$, called a **component of combined standard uncertainty** of y. The **combined standard uncertainty** of y, $u_c(y)$, is the positive square root of the sum of the squares of the components of combined standard uncertainty. The combined standard uncertainty is multiplied by a **coverage factor** of $k = 2$ to obtain U, the **expanded uncertainty** of y.

Since it can be assumed that the possible estimated values of the massic activity are approximately normally distributed with approximate standard deviation $u_c(y)$, the unknown value of the massic activity is believed to lie in the interval $y \pm U$ with a level of confidence of approximately 95 percent.

For further information on the expression of uncertainties, see references [2] and [3].

[d]   The value of each standard uncertainty component, and hence the value of the expanded uncertainty itself, is a best estimate based upon all available information, but is only approximately known. That is to say, the "uncertainty of the uncertainty" is large and not well known. This is true for uncertainties evaluated by statistical methods (e.g., the relative standard deviation of the standard deviation of the mean for the massic response is approximately 50%) and for uncertainties evaluated by other methods (which could easily be over estimated or under estimated by substantial amounts). The unknown value of the expanded uncertainty is believed to lie in the interval $U/2$ to $2U$ (i.e., within a factor of 2 of the estimated value).

[e]   The stated uncertainty is two times the standard uncertainty.

[f]   Estimated limits of detection for alpha-particle-emitting impurities are:
0.04 $\alpha \cdot s^{-1} \cdot g^{-1}$ for energies less than 3.9 MeV,
0.4 $\alpha \cdot s^{-1} \cdot g^{-1}$ for energies between 3.9 and 4.9 MeV, and
0.04 $\alpha \cdot s^{-1} \cdot g^{-1}$ for energies greater than 5.6 MeV.

[g]   Estimated limits of detection for photon-emitting impurities are:
0.013 $\gamma \cdot s^{-1} \cdot g^{-1}$ for energies between   9 and   125 keV,
0.009 $\gamma \cdot s^{-1} \cdot g^{-1}$ for energies between   135 and   234 keV,
0.004 $\gamma \cdot s^{-1} \cdot g^{-1}$ for energies between   243 and   579 keV,
0.002 $\gamma \cdot s^{-1} \cdot g^{-1}$ for energies between   588 and 1616 keV, and
0.001 $\gamma \cdot s^{-1} \cdot g^{-1}$ for energies between 1624 and 1900 keV,
provided that the photons are separated in energy by 4 keV or more from photons emitted in the decay of uranium-232 and progeny.

[h]   The stated uncertainty is the standard uncertainty.

[i]   Relative standard uncertainty of the input quantity x_i.

[j]   The relative change in the output quantity y divided by the relative change in the input quantity x_i. If $|\partial y / \partial x_i| \cdot (x_i/y) = 1.0$, then a 1% change in x_i results in a 1% change in y. If $|\partial y / \partial x_i| \cdot (x_i/y) = 0.05$, then a 1% change in x_i results in a 0.05% change in y.

[k]   Relative component of combined standard uncertainty of output quantity y, rounded to two significant figures or less. The relative component of combined standard uncertainty of y is given by $u_i(y)/y \equiv |\partial y / \partial x_i| \cdot u(x_i)/y = |\partial y / \partial x_i| \cdot (x_i/y) \cdot u(x_i)/x_i$. The numerical values of $u(x_i)/x_i$, $|\partial y / \partial x_i| \cdot (x_i/y)$, and $u_i(y)/y$, all dimensionless quantities, are listed in columns 3, 4, and 5, respectively. Thus, the value in column 5 is equal to the value in column 4 multiplied by the value in column 3. The input quantities are independent, or very nearly so. Hence the covariances are zero or negligible.

[m]   The relative standard uncertainty of $\lambda \cdot t$ is determined by the relative standard uncertainty of λ (i.e., of the half life). The relative standard uncertainty of t is negligible.

[n] $|\partial y / \partial x_i| \cdot (x_i/y) = |\lambda \cdot t|$

[p]   The live time is determined by counting the pulses from a gated oscillator.

[q]   The standard uncertainty for each undetected impurity that might reasonably be expected to be present is estimated to be equal to the estimated limit of detection for that impurity, i.e. $u(x_i)/x_i = 100\%$. $|\partial y / \partial x_i| \cdot (x_i/y) = \{(\text{response per Bq of impurity})/(\text{response per Bq of U-232})\} \cdot \{(\text{Bq of impurity})/(\text{Bq of U-232})\}$. Thus $u_i(y)/y$ is the relative change in y if the impurity were present with a massic activity equal to the estimated limit of detection.

[r]   More digits are shown than are significant in order to avoid roundoff errors and in order to see small differences in the ratios between the various progeny.

[s]   If the solution is disturbed, such as by opening the ampoule and dispensing or diluting the solution, some of the radon-220 will be lost from the solution and the progeny following radon-220 will have relative activities (in the solution) less than that shown in the table. If the solution (or aliquot) is resealed, the relative activities will increase back to their equilibrium values. The relative activities of the strongest gamma-ray emitters (lead-212, bismuth-212, and thallium-208) will decrease (and increase) with a half life approximately that of lead-212.

REFERENCES

[1]   International Organization for Standardization (ISO), *ISO Standards Handbook - Quantities and Units*, 1993. Available from Global Engineering Documents, 12 Inverness Way East, Englewood, CO 80112, U.S.A. Telephone 1-800-854-7179.

[2]   International Organization for Standardization (ISO), *Guide to the Expression of Uncertainty in Measurement*, 1993 (corrected and reprinted, 1995). Available from Global Engineering Documents, 12 Inverness Way East, Englewood, CO 80112, U.S.A. Telephone 1-800-854-7179.

[3]   B. N. Taylor and C. E. Kuyatt, *Guidelines for Evaluating and Expressing the Uncertainty of NIST Measurement Results*, NIST Technical Note 1297, 1994. Available from the Superintendent of Documents, U.S. Government Printing Office, Washington, DC 20407, U.S.A.

[4]   National Council on Radiation Protection and Measurements Report No. 58, *A Handbook of Radioactivity Measurements Procedures*, Second Edition, 1985. Available from the National Council on Radiation Protection and Measurements, 7910 Woodmont Avenue, Bethesda, MD 20814 U.S.A.

[5]   Evaluated Nuclear Structure Data File (ENSDF), July 2002.